Summary of MC Plasma

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on behalf of all contributors

18–22 Sept. 2017, Chengdu, China
Outline

— Highlight

— Major Progresses
  • Major progresses for machines, H-mode physics, ELM physics and control, energetic particles, MHD, Transport, Steady-state, PWI, Negative triangularity, diagnostics, Control&scenario development.

— Future Challenges

— Summary
General Information

Plenary (9), OV (20), I (69), O (31), P (63)

Experiments (121)

Theory (28)

Simulation (43)

Total: 192, Largest contribution in 1st DPP-AAPPS
Highlight

Significant progresses have been made
Efforts have been focus on physics understanding

- ITER on the right track, good progress for construction and preparation for operation (P4,OV11,OV16, I37)
- HL-2A explore robust ELM control methods (SMBI, RMP, LHCD, IM seeding, OV1)
- KSTAR strengthen the efforts for SS high beta plasma (OV2)
- LHD starts D operation (P25)
- EAST 100s H-mode (P11,OV6)

- Energetic particle remains the hot topic (20) and Nonlinear processes have been deeply addressed.
- Negative triangularity provide an alternative for power handling
- Small machines (15) make unique contributions for basic plasma science, physics understanding and training next generation plasma scientist.
- Theroy&Simulation (~40%) play a key role for understanding and future scenarios developments.
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— Summary
Asia leading devices provide major contributions

29 contributions

28 contributions

13 contributions

5 contributions
Efforts focus on plasma instability control and H-mode physics in HL-2A

- **Plasma instability control**
  - explored robust ELM control methods (SMBI, RMP, LHCD, IM seeding)
  - MGI+ SMBI to mitigation run away current
  - real-time active control of NTM by ECRH launcher mirror steering
  - suppression of ion fishbone by ECRH deliver low-Z gas for RE dissipation

- **H-mode physics**
  - Double critical gradients of electromagnetic turbulence in H-mode
  - mechanism of L-H transition
  - observed various pedestal instabilities

Using PAM launcher on H-mode plasmas

![Graph showing plasma parameters](image)

(P29) [MF-OV1] [MF-I3] [MF-O2] [MF-I29] [MF-05] [MF-I34] [MF-I36] [MF-I34] [MF-O20] [MF-I44] [MF-O26] [MF-O27] [MF-O30] [MF-I17] [MF-I19] [MF-O10] [MF-O12] +9 posters
The first deuterium experiment campaign was successfully finished on LHD

- The preparation and the commissioning for D-XP are proceeded successfully.
  - Calibration of Neutron diagnostic and Legal inspection completed, successfully.
  - Injection power of P-NBI is increased to 9MW. N-NBI decreases its injection power about a half due to the increase of co-extracted electron, i.e., isotope effect in negative-ion source.

- The first D-operation on LHD was quite successful.
  - The ion temperature of 10keV was achieved in D.-exp.
  - Some indication of isotope effects, electron energy transport and impurity behavior, were observed.
  - Neutron diagnostics accelerates useful EP confinement studies on helical machines.

Triton burn-up rate over 0.4% was achieved. Comparable to tokomak.

[ P25] [MF-14] [MF-19] [MF-159]
KSTAR made good progress in SS H-mode with high beta

- **Unique capabilities of KSTAR**: tokamak plasma symmetry, RMP coils, imaging diagnostics, and long pulse neutral beam.
- **Expanded operation regimes**: steady-state high beta operation (~72s) as well as alternative modes of high elongation (k >2), IBS, ITB, hybrid (G >0.4), high poloidal beta (β_p ~ 3), and low edge q.
- **ELM-crash suppression**: record long (~34s) ELM-crash suppression with n=1 RMP, and achieving suppression at ITER compatible low edge q (~3.4) with n=2 RMP.
- **Upgrade**: Plan is made which oriented to KDEMO

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Stationary high beta discharge (β_N ~ β_N ~ 2.8) at 0.4 MA, 3.9 MW NBI and 0.7 MW ECH

Record breaking long ELM-crash suppression (~34s)

[P18] [MF-Ov2] [MF-I27] [MF-I30] [MF-I55] [MF-I58] [MF-I45] [MF-I47] [MF-I16] [MF-I18] [MF-I51] + 2 posters
100s Steady-state H-mode operation with W-Wall

- ~101.2 s H-mode, $V_{\text{loop}} \approx 0$ V
- ~3 MW RF H&CD (LHW+ICRF+ECH)
- $H_{98y2} \approx 1.1$
- W-Divertor temperature was saturated after $t=20$ s
- $I_{\text{LHCD}}/I_p \approx 76$
- $f_{bs} \approx 23$
- $I_{\text{ECCD}}/I_p \approx 2.5$
Unique contribution from different machines

- **DIII-D** [Garofalo MF-OV12] [Marinoni MF-I54] [Humphreys MF-I35] [Tang MF-O25] [Xiao MF-I46] [Chen MF-O29]
  - Significant progress has been made in advancing the H-mode operating space and physics basis
  - QH-mode operation exceeds 80% of the Greenwald limit,

- **JET** [Romanelli MF-OV3]
  - Isotope experiments and scenario development: towards the DT phase
  - DD fusion yields have been extended to $2.9 \times 10^{16}$ neutrons/s for 5s

- **ASDEX-U** [Stober MF-OV7]
  - 20 MW of NBI, 7 MW of ICRF and 6 MW of ECRH equipped
  - Developed integrated scenarios for ITER and DEMO
Unique contribution from different machines

- **QUEST** [Hasegawa MF-OV10] [Onchi MF-I22] [Kuroda MF-I23]
  - Fully non-inductive plasma start-up, CHI
  - 1h55 min was successfully achieved, Control of hot wall
- **J-TEXT** [Yang MF-OV18]
  - RMP: an unique fast rotating capability (up to 6kHz)
  - Tearing mode control and disruption mitigation have been carried out
- **Heliotron-J** [Okada MF-OV14]
  - Using controllable five sets of coil systems to realizes a wide range of configurations by changing the coil-current ratios.
Unique contribution from different machines

- **TST-2** [Takase MF-OV4]
  - top-launch CCC antenna of lower hybrid wav alone demonstrated formation of an ST plasma

- **SUNIST** [Gao MF-OV19] [Tan MF-I69]
  - toroidal Alfven Eigen modes during minor disruptions have been found
  - Observed 3D structure of the eddy currents flowing in the split vacuum vessel

- **VEST** [Na MF-OV5]
  - Direct mode conversion of X-mode to Electron Bernstein Wave from the low field side is successfully utilized to enhance the ECH pre-ionization

- **TCV** [Porte MF-I53]
  - A significant improvement of the energy confinement time is observed in negative triangularity discharges
Unique contribution from different machines

- **KTX** [Liu MF-OV17]
  - Low q tokamak discharges up to 200kA with advanced diagnostics

- **RFX-mod** [Zuin MF-OV09]
  - Spontaneous helical equilibria formation associated to hot electron thermal structures
  - Energetic ion population self-generated in Ohmically heated RFP by magnetic reconnection

- **ADITYA/U** [Tanna MF-OV8]
  - low loop voltage start-up and current ramp-up experiments have been carried out using ECRH and ICRH

- **KMAX** [Sun MF-OV20]
  - A medium-sized washer gun is developed
  - Two ICRF systems can reliably deliver power ~100kW each
AAPPS should learn this from our EU colleagues.
H-mode Physics

QH, Small-ELM H-- For ITER and beyond

[A.Garofalo, MF-OV12] QH-mode at performance for Q=10 in ITER, High density 80% of the Greenwald limit

MF-OV12 A type of QH-mode obtained on EAST using tungsten divertor

[G.S.Xu, MF-I6] Stationary small-ELM H-mode regime operations in EAST

[A. Ekedahl, MF-I21] LHCD PAM for small ELM HL-2A

[L.M. Shao, MF-O11] The regime of small amplitude oscillations are consistent with the physical mode of zonal-flows and turbulence interaction at EAST.
H-mode Physics (L-H transition, regimes)

L-H Transition Studies under Non-axisymmetric Magnetic Fields in KSTAR [Won-Ha Ko, MF-I30]

Sensitive dependence on resonant components and no dependence of non-resonant fields on L-H power threshold

Validating gyrokinetic predictions

KSTAR 7s ITB discharge in a weakly reversed q-profile

Understanding confinement scaling \( t_E \sim 1/v \) at low collisionality critical for future STs [W. Guttenfelder MF-I11]

Zonal flows play a key role for L-H transition. The kinetic ballooning modes and peeling modes dominate on the H-mode barrier.
New shear flow oscillation observed in the pedestal
Shear flow oscillation and turbulence bursts are responsible for ELM mitigation and suppression.

[X.L. Zou MF-I21]
**ELM control by RMP**

Significant progresses have been made both on experiments and modeling

- Expanded operation boundary and capability of RMP-driven, ELM-crash-suppression  [Y. In, MF-I16]

[ L.Li MF-I20 ] [ M. Kim, MF-I18] modeling for understanding of ELM mitigation by RMP

[X. T. Yan MF-O12] RMP helicity could be used as a new scheme for controlling NTV peak location.

[S.Y. Liang MFP7] ELM mitigated by n=1 RMP in H-2A

- Magnetic topological change plays a key role in accessing final ELM suppression.  [Y.W. Sun, MF-I48]

[W.W. Xiao, MF-I46] Propagation Dynamics with Resonant Magnetic Perturbations Field in H-mode Plasmas

[J. K. Park, MF-I45] Optimization of Resonant and Non-resonant Magnetic Perturbations in KSTAR

[J. Cheng, MF-I19] Observation of streamer as a trigger of ELM in HL-2A
Energetic particle remains the hot topic (20)

Potential directions for EP studies
- AE transport of fast ions
  - Nonlinear regimes should be in focus both theoretically and experimentally
  - Initial value codes and reduced models are to be pursued
- Delterious effects on EP and thermal plasma confinement
- Ion Cyclotron Emission is important, partially (linearly) understood

[Subcritical instability of the GAM has been observed (parametric coupling and/or kinetic coupling)]

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[Interpretation]

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T. Ido MF-I4

Gorelenkov, MF-I1

LHD
**Energetic Particle Study--Experiments**

[L.M. Yu MF-I3] TAE with frequency in the range of 160-380 kHz is observed in high-power ECRH

[Y.B. Dong MF-I36] (TAE) was observed during the disruption, limiting the strength of runaway beam

[W. Chen MF-I56] AITG f=80-200 kHz (f_{BAE}<f<f_{TAE}) appears in the ITB plasmas with weak magnetic shear and high gradients of Ti;
Nonlinear simulation of TAE

Z. Qiu MF-I2 Nonlinear processes and saturated spectrum of Alfvén eigenmodes

Due to the typically radially localized TAE mode structures, the associated nonlinear processes also exhibit meso-scale; resulting in qualitative and quantitative modifications in the nonlinear processes of AEs.

H.S. Cai MF-I40
• Possibility to use EP to suppress NTM for the steady state and hybrid scenarios with weak magnetic shear.
• Influence of energetic ions on stability criterion of tearing modes, PRL 106

There are energy transport between double fishbones [J. Zhu MF-I66]
MHD instabilities-New findings

LHD

[K. Ida MF-I9]

- Tongue event triggers the minor plasma collapse
- A new trigger mechanism of MHD burst

[0.1] Various MHD modes have been observed in HL-2A

[Z.R. Wang MF-I41] Full toroidal computation of resistive MHD instabilities based on asymptotic matching approach

[Nornberg MF-I62] Using Integrated Data Analysis to optimize measurements critical to the validation of MHD simulations
MHD instability VS turbulence

— Multiscale nonlinear interaction between a large scale MHD instability and small scale turbulence
— Importance on the electron thermal transport

[M. J. Choi MF-I51]

[S. Sabbagh MF-I47] Generalized Neoclassical Toroidal Viscosity (NTV) Offset rotation profile $V_{0}^{NTV}$ measured in KSTAR, Potential aid for ITER.

[M. Jiang MF-O20] Radial profiles of poloidal flow and density fluctuation around the magnetic island were firstly observed on HL-2A tokamak
MHD instability-NTM

[H.S. Cai MF-I40] Provide a possibility of using EP to suppress NTM for the steady state and hybrid scenarios with weak magnetic shear.

[Z.X. Wang MF-I38] ECCD can effectively reduce 2/1 NTM island width. Deposition of ECCD at 2/1 surface can further stabilize 3/2 NTMs.

[F. Poli MF-I37] Simulations that evolve self-consistently NTM width and frequency and plasma profiles help designing more robust control schemes for ITER.

[H.P. Qu MF-I39] Magnetic islands and neoclassical currents

Stabilization of the 2/1 mode
MARS-Q modeling of a DIII-D ELM suppression experiment re-produces large density pump out [Y.Q. Liu MF-I33]

[D.Li MF-O18] Influence of strong magnetic field on plasma transport

[J. Weiland MF-I7] The role of zonal flows in reactive fluid closures

[Y. Ren MF-I15] GTS simulations demonstrate decent agreement in ion thermal transport

[SuminYi MF-I10] A gyrokinetic simulation study of Non-local Transport phenomenon

[Watanabe P5] MTM turbulence can be suppressed by ETGs through destruction of the current sheet structures

[Y. Xiao MF-I50] GTC simulation found the turbulent transport coefficient decreases with the applied gradient


[J.Q. Dong MF-I8] Impurity induced micro-electromagnetic instabilities in toroidal plasmas

Towards successful operation of ITER W divertor

Better understanding via modelling


[T.Y. Xia, MF-O24] Simulation of SOL width with helical current filaments

[C.F. Sang MF-I65] The closure of divertor also has great impact on the upstream plasma condition

[T. Wu MF-O23] Coupling of SOL density profiles with edge plasma parameters

[C.S. Corr MF-O19] High power, steady state MAGPIE II linear plasma device is under construction

Risk of melt damage if plasma operation fails to keep surface heat flux below thermo-mechanical limits.

Baseline is partially detached operation on full-W divertor using low Z seeding assist for dissipation of 50-70% \( P_{\text{SOL}} \)

[J.W. Coenen MF-I42] Baseline is partially detached operation on full-W divertor using low Z seeding assist for dissipation of 50-70% \( P_{\text{SOL}} \)
DSOL Physics, Plasma Wall Interaction

Control particle and heat fluxes in long pulse by integrated way

[J.S. Hu MF-I63] Li Experiments: effectively suppress W impurity

[L. Zakharov MF-I64] Plasma boundary play a key factor. FLiW is important for confinement and particle control.

[L. Wang MF-I43] Active handling of heat flux and impurity accumulation by utilization of in-out divertor asymmetry and optimization of configuration & strike point


[M. Francisquez MF-I31] Global 3D two-fluid tokamak edge simulations

K. Wu MF-O16] Active feedback control of radiation for power exhaust in EAST long-pulse operations
Efforts for steady state high performance

**EAST** [Y.F. Liang, MF-OV6]

All actively cooled PFC & Diagnostics, CW H&CD

**KSTAR** [Y.K. Oh, MF-OV2]

Fully non-inductive operation at $I_p = 0.8$ MA, $q_{95} = 5.3$

**ASDEX-U** full-W environment

**QUEST** [M. Hasegawa, MF-OV10]

non-inductive 1h55min

[Jörg Stober, MF-OV7]
Negative triangularity

- NTT reduce power load geometrically
- NTT stabilize TEM and can have HH~1.2 with L-mode edge

Pedestal in negative $\delta$ H-mode is smaller than in positive $\delta$

FTE coils ~4MA

[L. Porte MF-I53]

TCV

$\delta = -1$ $\delta = -1$ $\delta = +$ $\delta = +0.4$

[N. Kikuchi MF-013]

H-mode-like confinement with L-mode edge in negative triangularity plasmas on DIII-D

[A. Marinoni, MF-I54]

[VDEs investigation of the negative triangularity plasmas]

[L.Xue, MF-O14]
Plasma Diagnostics

**ITER diagnostics** [Michael Walsh, MF-I60]

- About 50 large scale diagnostic systems are foreseen:
  - Diagnostics required for protection, control and physics studies
  - Measurements from DC to $\gamma$-rays, neutrons, $\alpha$-particles, plasma species
  - Diagnostic Neutral Beam for active spectroscopy (CXRS, MSE ....)

**EAST**: [Liu MF-I61] Reliable all key profiles diagnostics for exploring high performance long pulse scenarios

**HL-2A**: [Yuan MF-O10] [Liu MF-O26]—A new gas-puff imaging (GPI) diagnostic system has been developed to study two-dimensional (2-D) plasma turbulence—$\text{Z}_{\text{eff}}$ measurement by visible bremsstrahlung diagnostic

**HL-2A**: 7-channel MSE diagnostics based on dual PEMs [Chen MF-O27]

**RELAX**: high-speed tangential SXR imaging diagnostics were developed to identify the emission structures [A. Sanpei MF-O7]
Operation control and scenarios development

DIII-D, EAST, KSTAR are advancing integrated plasma control toward disruption-free operation

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KSTAR upgrade

off-axis NBI ~ 6 MW (2019) and ECH ~6 MW & in-vessel components to address critical issues at high beta steady-state operation for k-DEMO realization (\(b_N \sim 4.0\), \(f_{BS} \sim 0.7\), \(f_{GW} \sim 1\) with long-pulse steady-state conditions)

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Addressing Physics issues in operation conditions expected on ITER, CFETR. High \(b_N\), Ti \~ Te, and vanishing loop voltage simultaneously

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\[ D.A.~Humphreys~MF-I35 \]

\[ S.Ding~MF-I28 \]

\[ X.Q.~Ji~MF-I34 \]

\[ Joerg~Stober~MF-OV07 \]

\[ M.~Romanelli~MF-I26 \]
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Future Challenges
Outstanding Issues with Gaps beyond ITER

Steady-state H operation (weeks)
most novel part of DEMO
Core
Efficient non-inductive CD in H-mode
High bootstrap current fraction
Low impurity concentration

Edge
Controllable PSI for lowering impurity
generation and particle recycling in W divertor
Low peak heat load Tolerable transient heat
shock (small/no ELMs)
erosion free

Power Exhaust
Peak heat fluxes near technological limits
(>20 MW/m²)
ITER solution may be marginal for DEMO
Integration of DEMO working condition
is very challenging

Need both new physical
(advanced divertor
+impurity seeding) and
technical (new robust
DEMO 20MW/m²
target) solutions
NTT reduce power
load geometrically by
factor of 7?
Validation on long pulse
tokamak experiments.

[B.N Wan, P11]
Future Challenges

Outstanding Issues with Gaps beyond ITER

**Off normal events**
- Exits for nearly 50 years and not solved yet.
- Efforts for theory & modeling and technology for advanced integrated control to a Robust, Disruption-Free Operation
- Experimental validation to 100% c

**Burning Plasma**
- Confinement & transport for alpha particle heating dominated plasma in the presence of AE driven by superthermal fast ions (MF-I1)
- Significant loss of Alpha/fast ions degrades plasma H&CD efficiency, may quench DT burning (P2-2)
- Robust, simple burning plasma diagnostics. Lesson learned from ITER (MF-I60)
Summary

- Significant progresses have been made in DPP-AAPPS. Efforts have been which focus on physics understanding.
- Understanding of H-mode & ELMs, and effective control scenarios have been progressed.
- Transport / turbulence / instabilities are reproduced well by simulations.
- Energetic particles and advanced SSO remain challenging and efforts should be made for near future.
- Fusion is a century project which involves science, technology and engineering. Scientifically, we have to make targets more simple rather than more complicated. Technically, we have to make every component and system robust and reliable towards our final goal. Let's work on it.
- Training excellent young talent is very important. Please bring more students to next DPP meeting.
See you in Japan