

8th Asia-Pacific Conference on Plasma Physics, 3-8 Nov, 2024 at Malacca **nHESEL predictions of density shoulder formation in JET-ILW H-, D-, T-isotope L-mode plasmas**

A.S. Thrysøe¹, A.H. Nielsen¹, M. Groth², J. Juul Rasmussen¹ and JET contributors*

¹ Department of Physics, Technical University of Denmark, ²Aalto University, Espoo, Finland, *See the author list of "Overview of T and D-T results in JET with ITER-like wall" by C.F. Maggi et al. to be published in Nuclear Fusion Special Issue: Overview and Summary Papers from the 29th

Fusion Energy Conference (London, UK, 16-21 October 2023)

e-mail (speaker): alec@dtu.dk

nHESEL simulations of L-mode JET ITER-like wall (JET-ILW) plasma edge and scrape-off layer (SOL) regions in protium, deuterium and tritium scenarios predict that the power flux across the separatrix scales inversely with the mass number A when the edge profiles are kept fixed across isotope species. This is consistent with experimental observations [1] and a desired feature for operations on future D-T devices such as ITER. Simulations with a higher A also show an increase in the density and power fall-off length in the outer midplane SOL, potentially resulting in an undesired increase of plasma-wall main-chamber interactions. The characteristics of the density shoulder are shown to depend on the divertor detachment scheme, interactions with neutral particles, and other plasma parameters such as electron density.

nHESEL is a drift-reduced two-fluid code that simulates the electron and ion densities and pressures, and the generalized vorticity, in a two-dimensional cross-field domain at the outer midplane of JET [2,3]. The effects of downstream conditions are included in the parameterized The parallel loss terms. plasma fields are self-consistently interacting with dynamic hydrogen isotope neutrals modelled as a multi-component fluid consisting of molecules, and Franck-Condon and charge-exchanged atoms.

It is demonstrated that the upstream simulations can estimate the power deposition profiles downstream, i.e., at the divertor targets for the different hydrogen isotopes. The heat flux to the divertor targets is another constraint that sets the required degree of detachment in ITER and future fusion power plants. The power fluxes predicted by the numerical simulations are consistent with those measured in JET and to those simulated with EDGE2D-EIRENE, [4]. The consistency gives confidence in the application of the model as a synthetic diagnostic, not only for local measurements at the outer midplane but also for downstream features such as target heat load profiles.

The increase in power flux across the separatrix with lower A is well understood from its explicit dependence in the instability-driving part of the generalized vorticity equation. The midplane SOL profiles are determined by the balance between cross-field and parallel transport, and by the volume sources from local interactions with neutrals. The larger fall-off length for larger A is mainly attributed to the lower parallel speed of the heavier isotopes.

References

[1] C.F. Maggi et al. 2018 Plasma Phys. Control. Fusion 60 014045

[2] A.H. Nielsen et al., Phys. Letters Section A, **379** 3097–3101 (2015)

[3] A.S. Thrysøe et al., Physics of Plasmas, Phys. Plasmas **25**, 032307 (2018)

[4] M. Groth et al, Nuclear Materials and Energy 34 (2023) 101345